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Implementation and Verification of Element-Wise Density and Temperature Specifications in MCNP6 Unstructured Mesh Simulations

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"The Stanford 3D Scanning Repository", graphics.stanford.edu. Retrieved June 2024. Converted into an unstructured mesh.

Errors related to spatial resolution are often overlooked in nuclear simulations

- Even if the geometry is modeled with a high spatial resolution, material properties are often modeled at a much coarser spatial resolution and this error is sometimes neglected
 - This work is intended to reduce this error in MCNP simulations



The motivation for this work is to generate higher-fidelity multi-physics MCNP simulations

- The ability to specify element-wise material densities and temperatures in MCNP unstructured mesh simulations was recently added (and will supported in MCNP6.4)
 - "Element" refers to an individual tetrahedron, pentahedron or hexahedron in an unstructured mesh
 - MCNP users will no longer need to average densities and temperatures on a much coarser spatial resolution for multi-physics simulations



The MCNP UM HDF5 file was modified to include element-wise densities and temperatures

- The material group in the new MCNP UM HDF5 file must now contain the following attributes and datasets
 - material_id: a 1D dataset composed of a single integer
 - density_by: an ASCII string (either "element" or "pseudocell")
 - density: a 1D dataset of non-negative real numbers
 - **temperature_by**: an ASCII string (either "element" or "pseudocell")
 - temperature: a 1D dataset of non-negative real numbers

The MCNP UM HDF5 format in this presentation is more recent than the format described in the ANS summary and is likely the format that will be required in MCNP6.4



If there are element-wise differences in material properties, particle tracking behavior is modified

If there are no element-wise differences



If there **are** element-wise differences



- → represents particle tracks
- ☆ represents material property look-ups & resampling of distance-to-collision



Element-wise material specification was verified using three test problems

- Implementation was verified using:
 - two analytical test problems
 - one code-to-code verification



Analytical test problem 1: A cube (1 cm³) with density varying in the x-dimension



Analytical result for volume-averaged flux (F4) for the cube:

$$\overline{\phi} = \int_{0}^{1} dx \exp\left(-\int_{0}^{x} dx' \Sigma_{a}(x')\right) \approx 0.924023$$



Analytical test problem 1 convergence study: simulation error as function of mesh resolution

Mesh Size (cm)	Number of Tetrahedra	Disagreement \pm Noise	10^{-2}	+	
1	12	1.3×10 ⁻² ± 2×10 ⁻⁴			
0.5	49	5.1×10 ⁻³ ± 2×10 ⁻⁴			
0.25	430	9.7×10 ⁻⁴ ± 2×10 ⁻⁴	на н		
0.125	4051	1.8×10 ⁻⁴ ± 2×10 ⁻⁴	10^{-4}		
$10^{-1} 10^{0}$					

- **Disagreement:** relative error between MCNP and analytical result
- Noise: one standard deviation of statistical uncertainty in MCNP ٠



Analytical test problem 2: A cube (1 cm³) with temperature varying in the x-dimension



Analytical result for volume-averaged flux (F4) for the cube:

$$\overline{\phi} = \int_{0}^{1} dx \exp\left(-\int_{0}^{x} dx' \Sigma_{a}(x')\right) \approx 0.780968$$



Analytical test problem 2 convergence study: simulation error as function of mesh resolution



- **Disagreement:** relative error between MCNP and analytical result
- Noise: one standard deviation of statistical uncertainty in MCNP



Code-to-code verification problem: UM implementation versus equivalent CSG



- 26056.00c ACE file used as room-temperature cross-section data
- Four MCNP tallies are used:
 - F1: a surface current tally on the right boundary, x=4
 - F2: a surface flux tally on the right boundary, x=4
 - F4: a volume-averaged flux tally for all four cubes
 - F6: an energy deposition tally for all four cubes
- Only 1000 particle histories for each simulation



Code-to-code verification problem results: UM and CSG simulations were identical

Tally	Geometry	Mean \pm One Standard Deviation
F1	CSG	2.5867×10 ⁻¹ ± 1.3839×10 ⁻²
	UM	2.5867×10 ⁻¹ ± 1.3839×10 ⁻²
F2	CSG	2.5867×10 ⁻¹ ± 1.3839×10 ⁻²
	UM	2.5867×10 ⁻¹ ± 1.3839×10 ⁻²
F4	CSG	7.6052×10 ⁻¹ ± 9.2023×10 ⁻³
	UM	7.6052×10 ⁻¹ ± 9.2023×10 ⁻³
F6	CSG	5.6952×10 ⁻⁷ ± 9.9096×10 ⁻⁹
	UM	5.6952×10 ⁻⁷ ± 9.9096×10 ⁻⁹



Implementation of element-wise material properties was also tested using the MCNP test suite

- The MCNP test suite verified that new feature is compatible with other existing code features
- A few new tests related to element-wise material properties were added to the test suite



Conclusions

- Element-wise density and temperature specification were successfully implemented in the MCNP code
 - In MCNP6.4, a user will be able to specify element-wise densities and temperatures in the MCNP UM HDF5 file
 - Two analytical verification problems demonstrated mesh convergence for MCNP simulations with element-wise densities and temperatures
 - A code-to-code comparison verified that MCNP particle-tracking and tallying are agnostic to the geometry system (CSG/UM)
 - The code implementation was also verified by the MCNP test suite



Future Work

- Investigating MCNP computational performance and memory use for simulations with element-wise densities and temperatures
- Testing element-wise densities and temperatures with more complicated geometries, including problems with mixed element types
- Extending the MCNP code's UM capability to also enable element-wise isotopic compositions



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